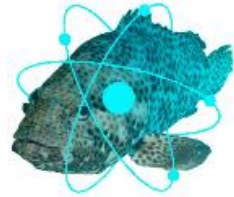


CHERNE 2016

12th Workshop on European Collaboration for Higher Education and
Research in Nuclear Engineering and Radiological Protection
30 May – 1 June 2016 Cervia



Efficiency calibration of a portable CZT detector for non-destructive activation assessment of a cyclotron bunker

S. Vichi¹, A. Infantino¹, F. Zagni², G. Cicoria², M. Marengo², D. Mostacci¹

¹*Montecuccolino Nuclear Engineering Laboratory, Department of Industrial
Engineering, University of Bologna, Bologna, ITALY,*

²*Medical Physics Department, “S. Orsola-Malpighi” Hospital, Bologna, ITALY*

sara.vichi3@unibo.it

Introduction

- The use of accelerators in the medical field has grown significantly in the last two decades.
- The estimated life expectancy, as well as the reasons for shutting down accelerators are almost as diverse as the type of accelerators present :
 - financial or political issues;
 - market evolution;
 - technological improvements:
 - changes in institution goals;
 - aging of equipment;
 -



^{18}F ^{15}O
 ^{11}C  ^{13}N
 +
 Non-standard radionuclides

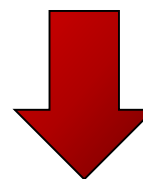
According to IAEA in the world there are actually 15000 accelerators installed, of which only a few hundred for research while 97% are used for medical and industrial application

Introduction

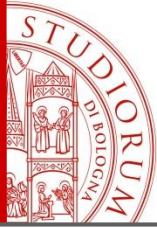
In Italy there are 36 PET cyclotrons, most of these are in use by 10-15 years.



During the operational life of a PET Cyclotron, the concrete walls of the cyclotron vault are activated by the secondary neutron flux interacting with rare earth's and metals present in the concrete or in reinforcement bars



when considering dismantling of such accelerators, the amount of radioactive waste has to be preventively evaluated to identify any critical issues or possible countermeasures to be taken in order to define an **optimum decommissioning strategy**



Introduction

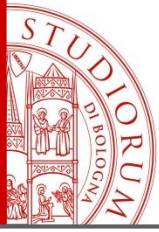
Critical Aspects

As long as an accelerator is operational, experimental measurements inside the bunker are problematic for different reason:

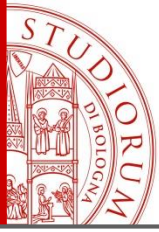
- Access time in bunker is limited;
- The radioactive background due to activated material of cyclotron itself is high;
- Presence of short-lived radionuclides with an activity concentration significantly higher respect to activity concentration of long-lived radionuclides;
- Impossibility to perform core drilling;



Aim of this work is to define a non-destructive in-situ measurement methodology for a preliminary activation assessment of a cyclotron bunker without the need of core drilling.

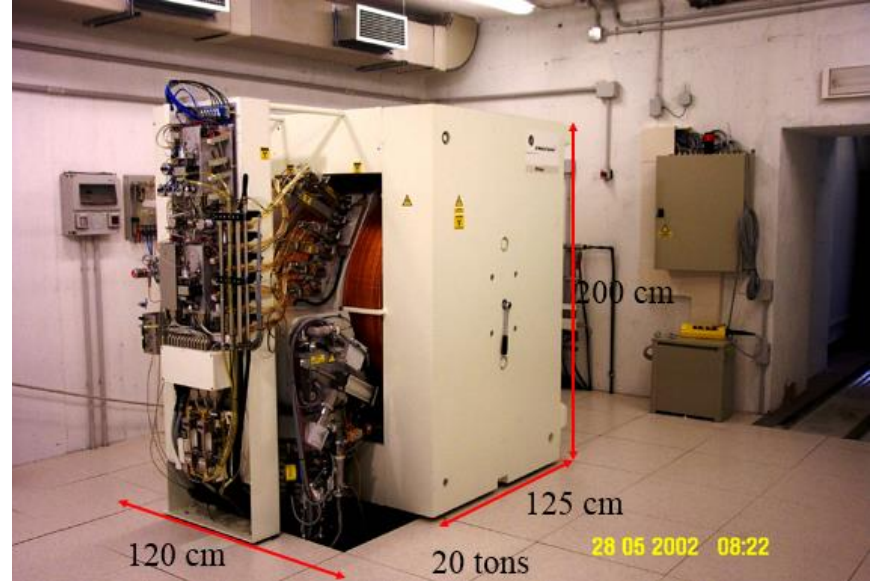


Material and methods



S. Orsola-Malpighi University Hospital

Medical Physics Department,
S. Orsola-Malpighi Hospital, Bologna



The GE PETtrace cyclotron (16.5 MeV) installed at “S. Orsola-Malpighi” University Hospital (Bologna, IT).

Typical daily production irradiation is made for 60 min with beams currents of 60 μA .

Detector: Kromek GR1



Specification

- **Detector:** 10mm x 10mm x 10mm CZT detector
- **Energy range:** 30 keV to 3.0 MeV
- **Energy resolution:** <2.5% FWHM @662 keV
- **Number of channels:** 4096 (12 bit)
- **Dimensions:** 25mm x 25mm x 63mm
- **Weight:** 60 gram

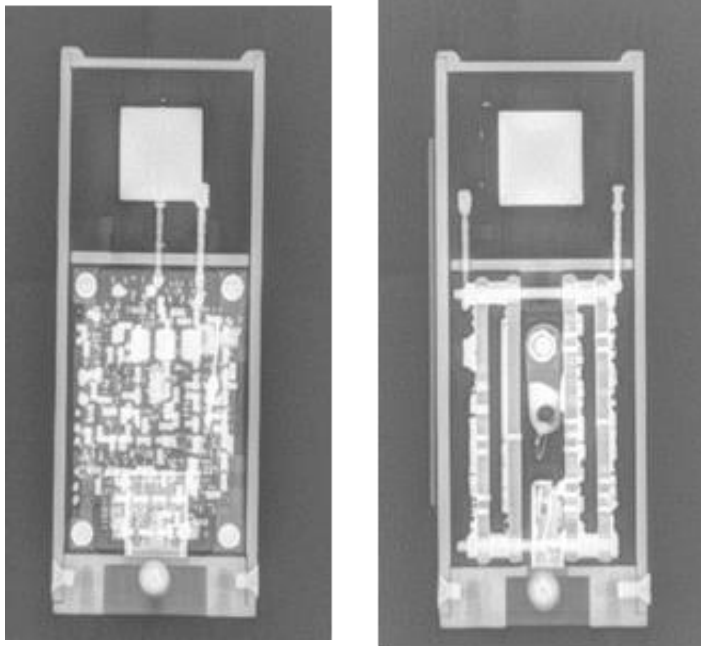


Critical Aspect:

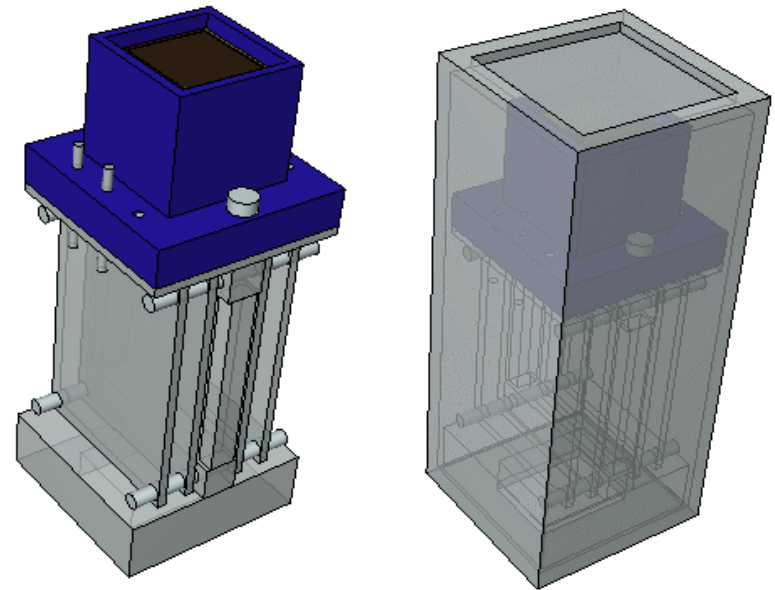
Efficiency calibration in non-standard geometry



Monte Carlo Model of the detector using FLUKA



Radiography of the detector



3D Model of the detector

FLUKA Input parameters

DEFAULTS: EM-CASCAdE

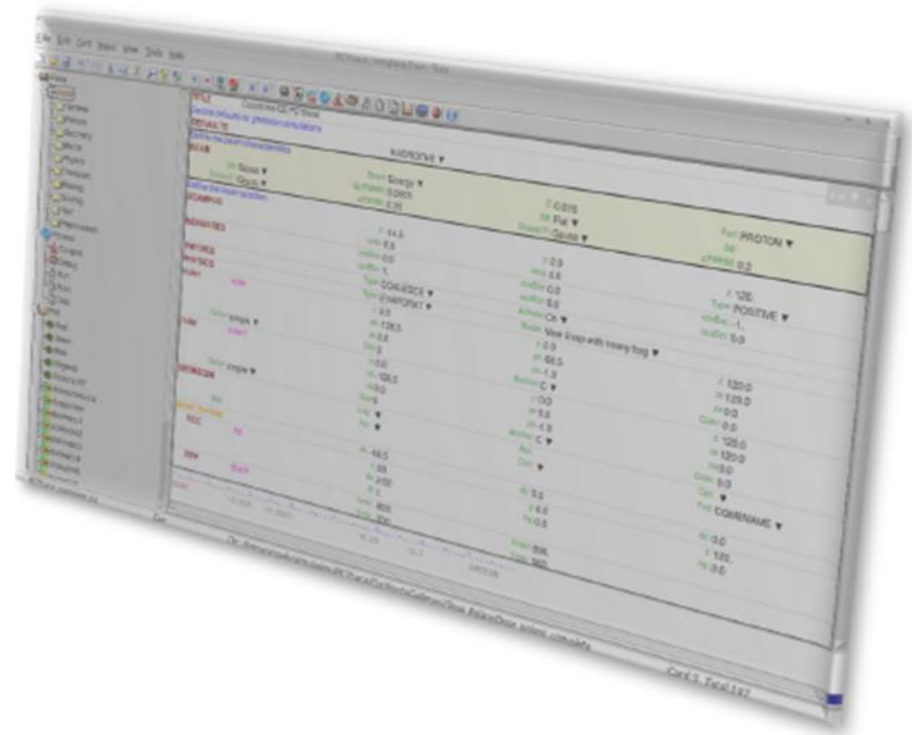
- Electromagnetic interactions ON
- Delta ray production enabled
- Heavy particle bremsstrahlung activated with explicit photon production above 1 MeV
- Heavy particle e⁺/e⁻ pair production activated with full explicit production

PHYSICS & TRANSPORT

- Energy thresholds for electron and photon production 1 keV
- Electron and photon transport cutoffs 1 keV

SCORING

- Photopeak Efficiency (DETECT)

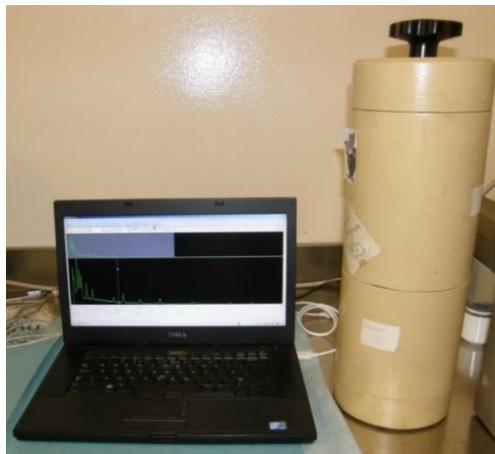


Validation of the MC model in standard geometry

Experimental measurements

➤ Point source:

^{57}Co ^{139}Cr ^{51}Cr ^{113}Sn ^{85}Sr
 ^{137}Cs ^{88}Y ^{60}Co



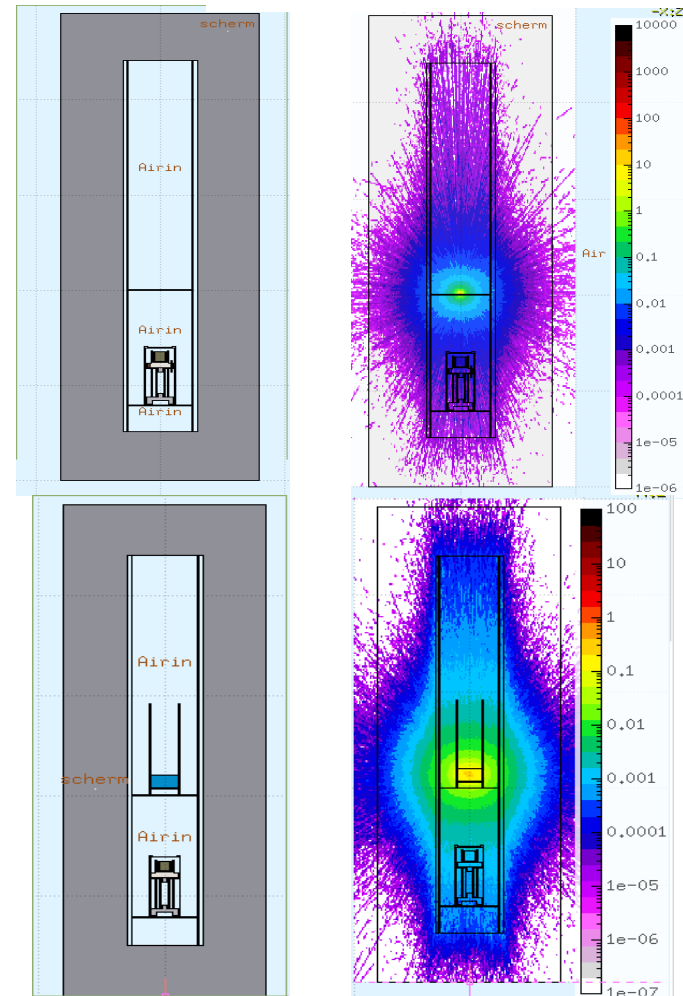
➤ 5cc Vial:

^{57}Co ^{137}Cs
 ^{60}Co



VS

Simulations

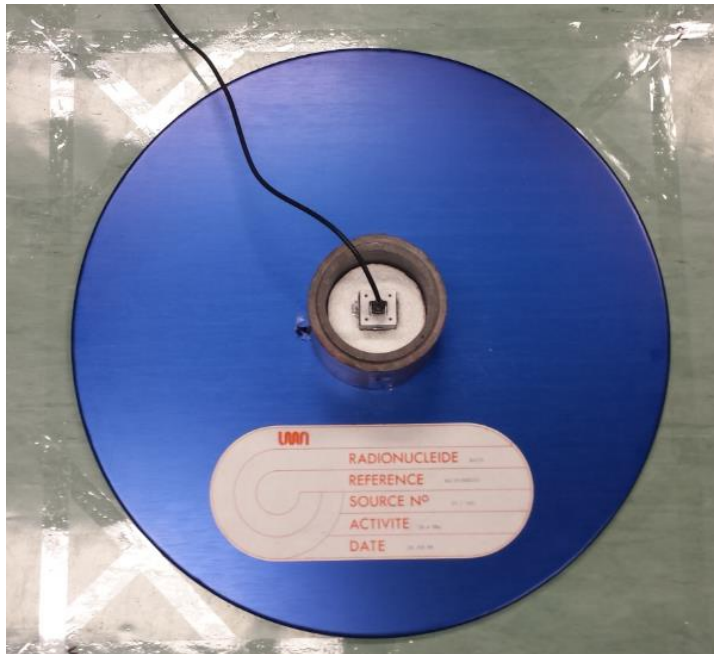


Validation of the MC model in standard geometry

Experimental measurements

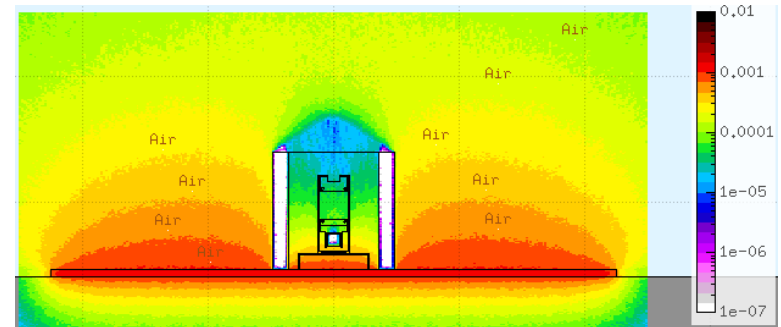
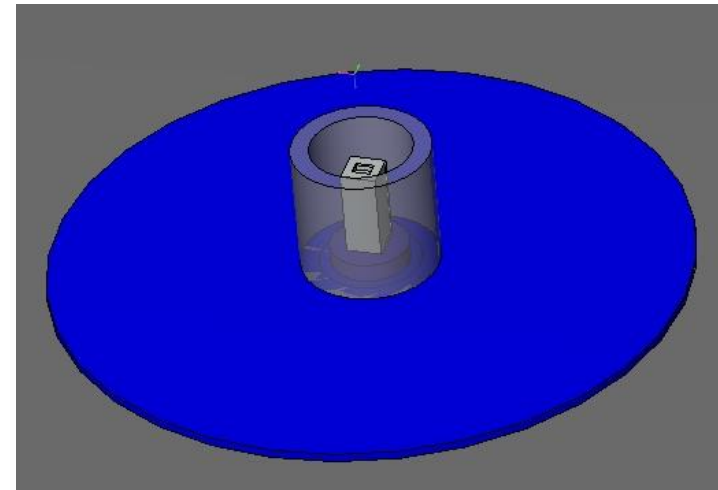
- **Extended source of ^{133}Ba (22 cm radius)**

(80 keV; 302 keV; 356 keV; 383 keV; 276 keV)

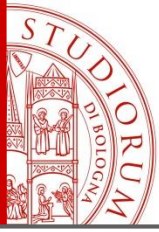


VS

Simulations

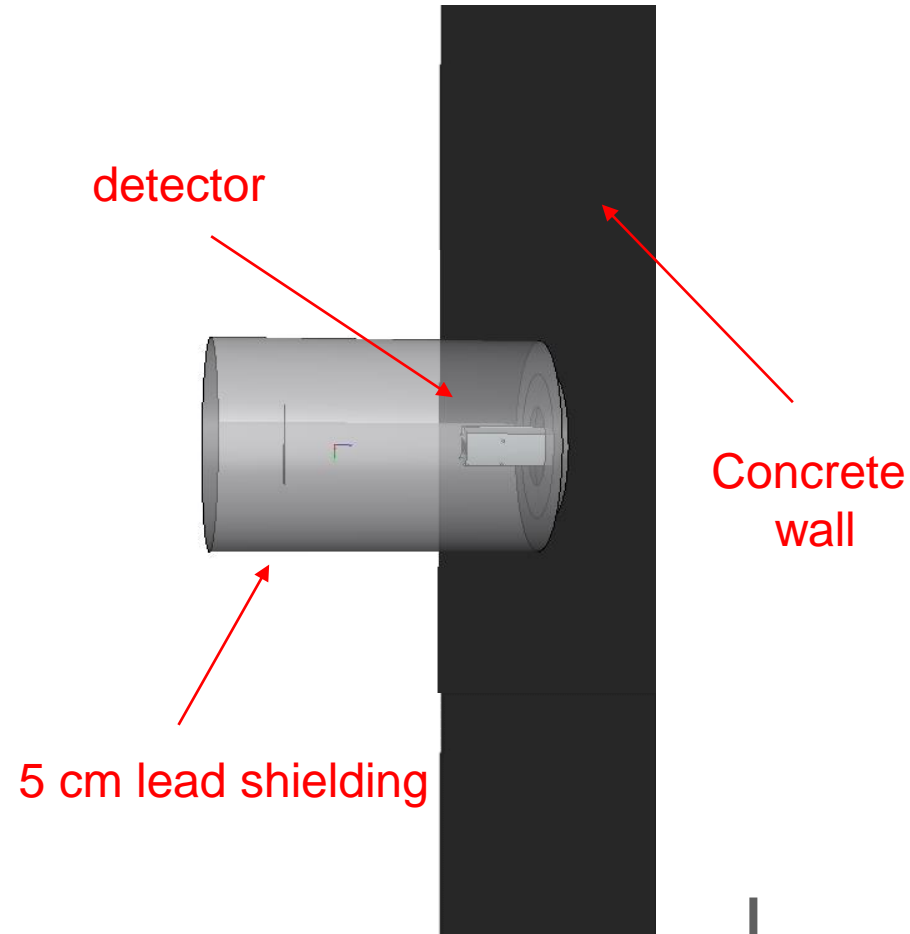


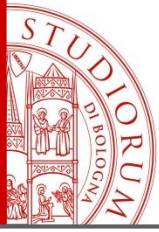
Photons/ cm³ per primary



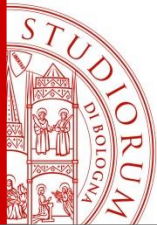
Efficiency calibration in non-standard geometry

- **Source:** concrete wall uniformly activated
- **Primary particles:** photons (122.06 keV; 165.86 keV; 320.08 keV; 344.27 keV; 391 keV; 514.00 keV; 661.66 keV; 847 keV; 898.04 keV; 1173.23 keV; 1332.49 keV)
- **N° of primaries simulated:** 1×10^9
- **Processor:** i7-4790
- **N° core:** 4 (8 thread)
- **N° runs:** 8
- **N° cicle:** 5





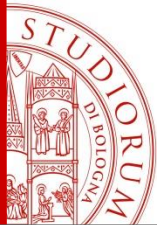
Results



Validation of the MC model

Point source

		Experimental measurements		FLUKA		
Nuclide	Energy (keV)	efficiency	Uncertainty (%)	efficiency	Uncertainty (%)	FLUKA/experimental
⁵⁷ Co	122.06	1,21E-03	1.56	1,21E-03	0.91	0.997
¹³⁹ Cr	165.86	1,11E-03	1.59	1,10E-03	1.09	0.993
⁵¹ Cr	320.08	3,94E-04	1.76	3,91E-04	1.60	0.992
¹¹³ Sn	391.70	2,56E-04	1.60	2,59E-04	2.20	1.010
⁸⁵ Sr	514.00	1,67E-04	1.71	1,57E-04	2.81	0.944
¹³⁷ Cs	661.66	9,89E-05	1.62	9,91E-05	3.18	1.000
⁸⁸ Y	898.04	5,50E-05	1.77	5,83E-05	4.63	1.060
⁶⁰ Co	1173.23	3,31E-05	1.76	3,36E-05	1.72	1.020
⁶⁰ Co	1332.49	2,92E-05	1.79	2,97E-05	5.80	1.020
⁸⁸ Y	1836.05	1,57E-05	2.24	1,59E-05	6.75	1.010



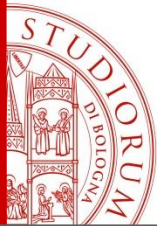
Validation of the MC model

5cc Vial Source

		Experimental measurements		FLUKA		
Nuclide	Energy (keV)	efficiency	Uncertainty (%)	efficiency	Uncertainty (%)	FLUKA/experimental
⁵⁷ CO	122.06	7.67E-04	1.92	7.35E-04	1.17	0.958
¹³⁷ CS	661.66	6.09E-05	1.64	6.33E-05	3.98	1.040
⁶⁰ CO	1173.23	2.42E-05	1.82	2.41E-05	2.04	0.996
⁶⁰ CO	1332.49	1.99E-05	1.88	2.03E-05	2.21	1.020

¹³³Ba Extended Source

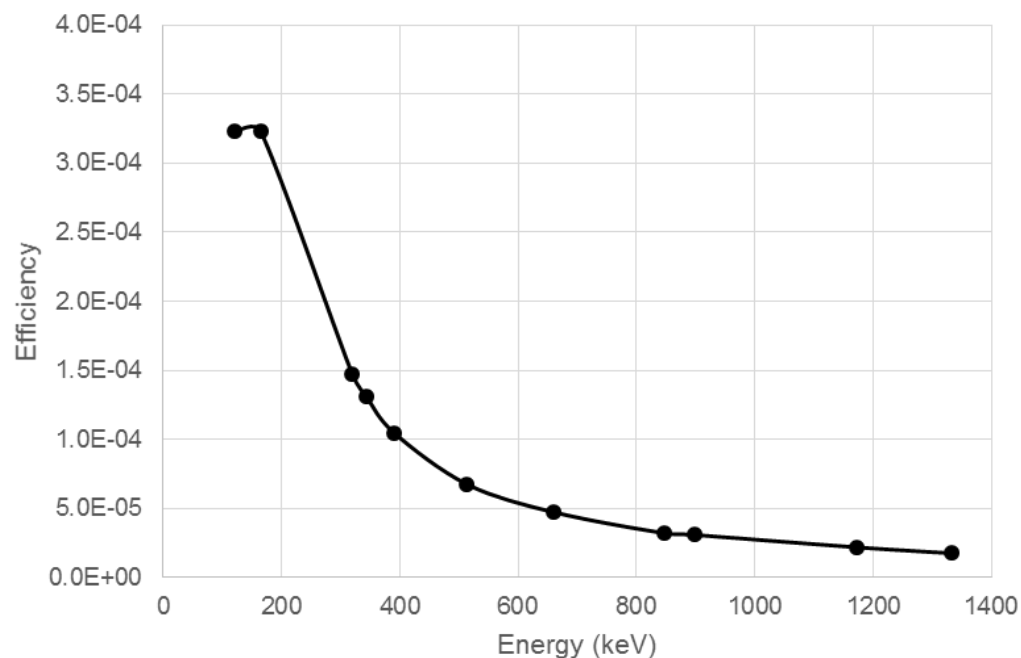
		Experimental measurements		FLUKA		
Energy (keV)	efficiency	Uncertainty (%)	efficiency	Uncertainty (%)	FLUKA/experimental	
276.40	5,40E-05	4.10	5.63E-05	1.33	1.040	
302.85	4,76E-05	4.04	4.95E-05	1.42	1.040	
356.02	3,81E-05	1.55	3.80E-05	1.62	0.995	
383.85	3,31E-05	1.71	3.32E-05	1.83	0.999	



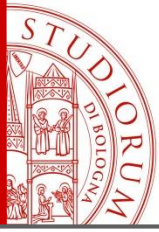
Efficiency calibration curve for wall activation assessment

Concrete wall

Energy (keV)	efficiency	Uncertainty (%)
122.06	3.23E-004	0.6
165.86	3.23E-004	0.6
320.08	1.47E-004	0.8
344.27	1.31E-004	0.9
391	1.04E-004	1.0
514.00	6.72E-005	1.2
661.66	4.71E-005	1.5
847	3.19E-005	1.8
898.04	3.07E-005	1.8
1173.23	2.17E-005	2.1
1332.49	1.74E-005	2.4



The efficiency curve for wall activation measurements was calculated, allowing a quantitative evaluation of activity concentration.



Experimental measurements: long-lived radionuclides

^{152}Eu (344 keV)

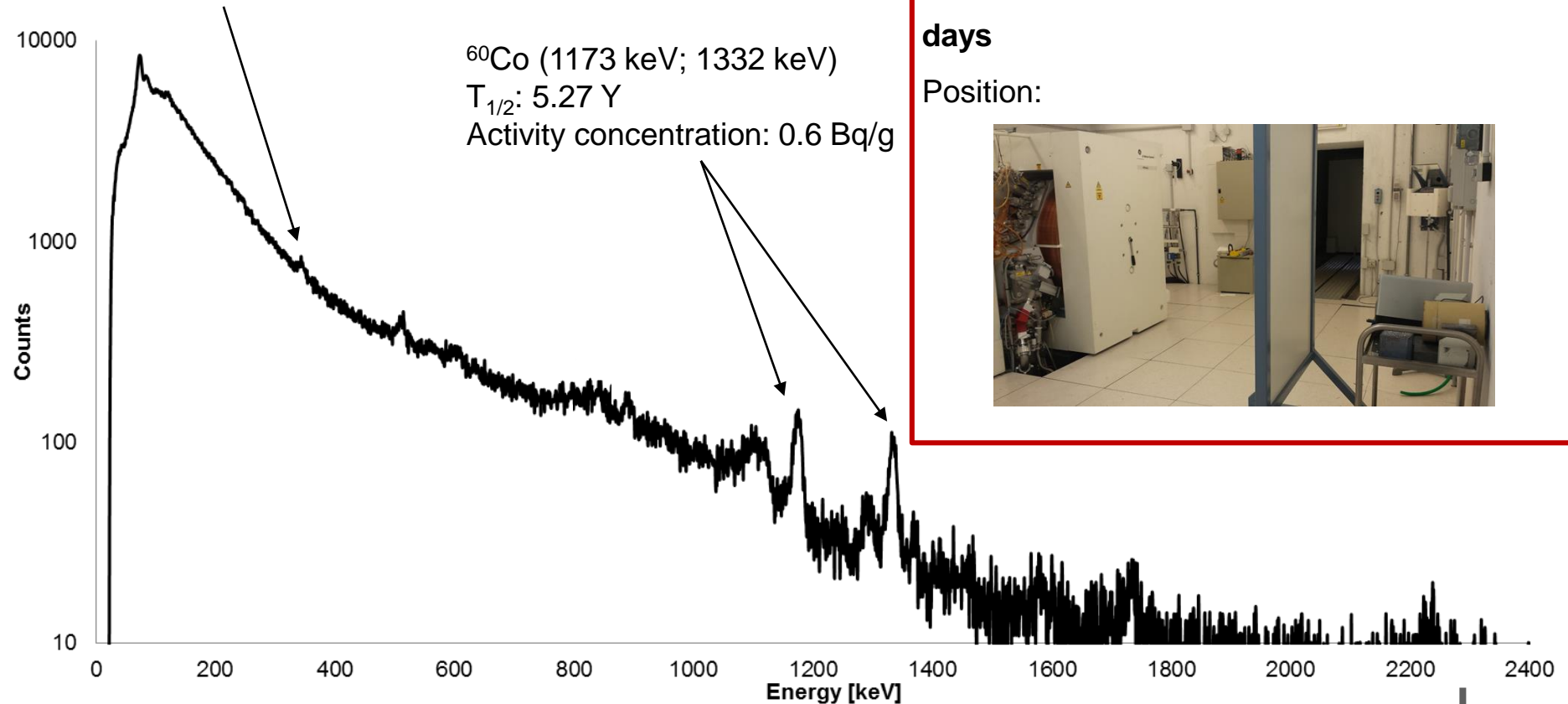
$T_{1/2}$: 13.5 Y

Activity Concentration < 0.1 Bq/g

^{60}Co (1173 keV; 1332 keV)

$T_{1/2}$: 5.27 Y

Activity concentration: 0.6 Bq/g

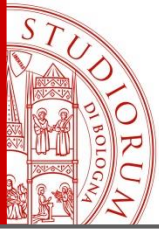


Acquisition time: **15 h during maintenance downtime**

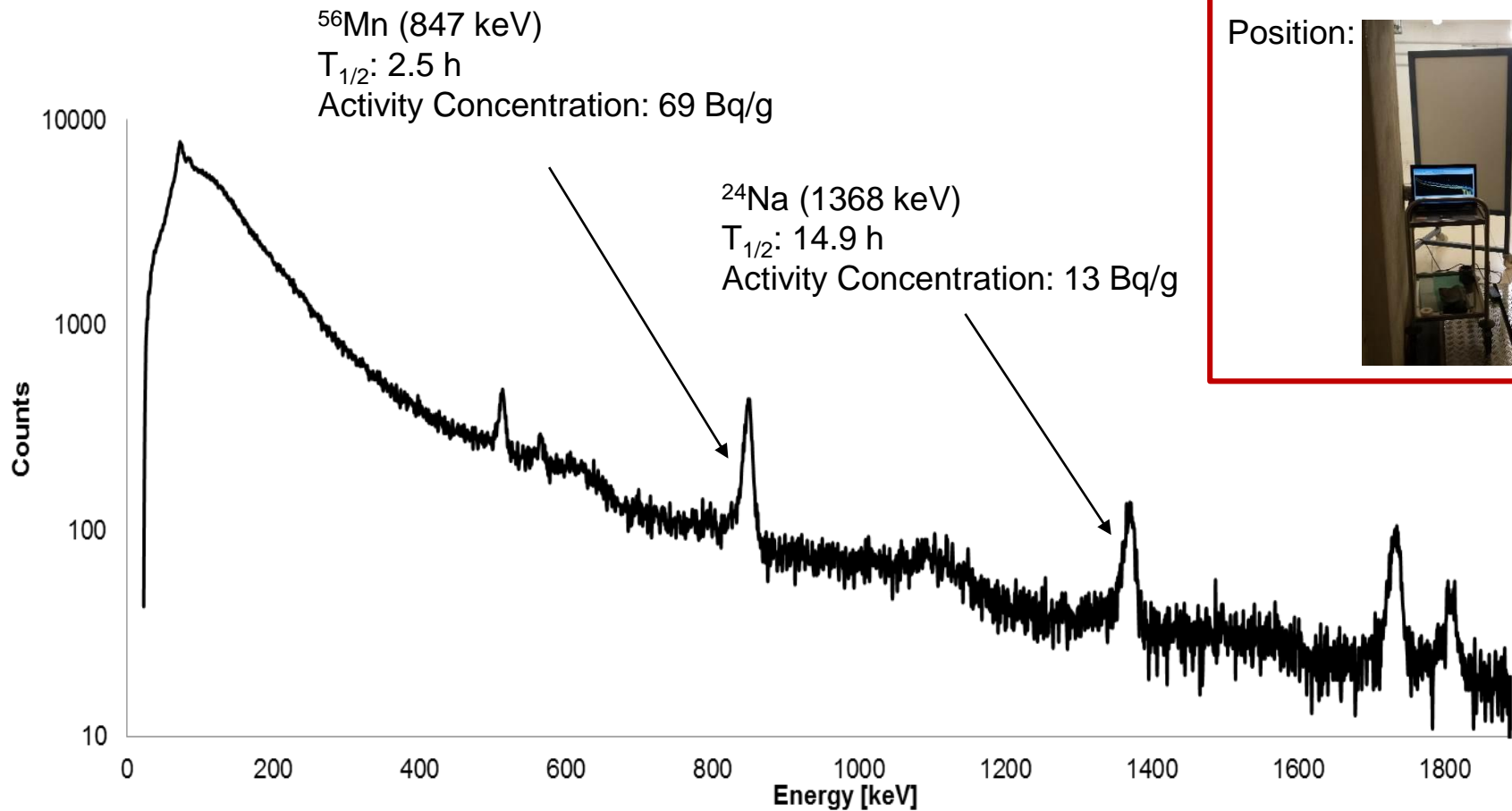
Time from the end of bombardment (EOB): **4 days**

Position:





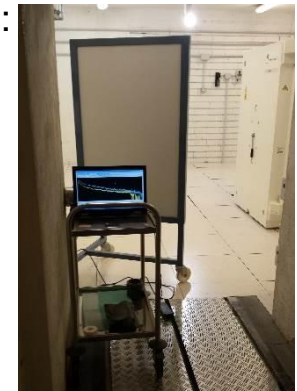
Experimental measurements: short-lived radionuclides

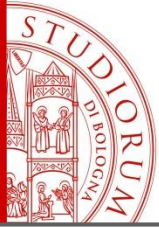


Aquisition time: 1 h

Time from the EOB: 5 h

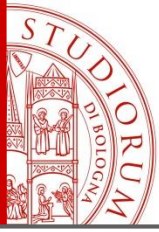
Position:





conclusions

- a non-destructive in-situ measurement methodology with a compact and portable CZT detector was developed;
- this methodology can be applied in the study of cyclotron sites, but also to radiotherapy linacs and other accelerators.
- Monte Carlo simulation proved to be a powerful tool for assessment of efficiency calibration curves for portable instruments used on the field in non-standard geometry conditions.



Thanks for your attention!

